

ACCESSION #: 9611190094

LICENSEE EVENT REPORT (LER)

FACILITY NAME: Vogtle Electric Generating Plant - Unit 2 PAGE: 1 OF 3

DOCKET NUMBER: 05000425

TITLE: REACTOR TRIP DUE TO MAIN FEEDWATER REGULATING VALVE CLOSURE

EVENT DATE: 10/14/96 LER #: 96-006-00 REPORT DATE: 11/08/96

OTHER FACILITIES INVOLVED: DOCKET NO: 05000

OPERATING MODE: 1 POWER LEVEL: 053

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR SECTION:

50.73(a)(2)(iv)

LICENSEE CONTACT FOR THIS LER:

NAME: Medhi Sheibani, Nuclear Safety TELEPHONE: (706) 826-3209

and Compliance

COMPONENT FAILURE DESCRIPTION:

CAUSE: B SYSTEM: SJ COMPONENT: PCV MANUFACTURER: B045

REPORTABLE NPRDS: Y

SUPPLEMENTAL REPORT EXPECTED: NO

ABSTRACT:

On October 14, 1996, power ascension was in progress following a refueling outage. The steam generator (SG) #3 main feedwater regulating valve (MFRV) went from being mostly open to the 25 percent open position and SG #3 water level began to decrease. Personnel were unsuccessful in attempts to re-open the MFRV. At 1925 EDT, SG #3 level had dropped to 43 percent (narrow range) and a manual reactor trip was initiated. Isolation of the main feedwater system and actuation of the auxiliary feedwater system occurred as designed. Operators stabilized SG water levels and transitioned the unit to normal operation in Mode 3 (hot standby).

The cause of this event was a failure of the MFRV positioner mechanism. The positioner was replaced and the valve was returned to service.

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#### A. REQUIREMENT FOR REPORT

This report is required per 10 CFR 50.73 (a)(2)(iv) because an unplanned reactor protection system actuation occurred.

#### B. UNIT STATUS AT TIME OF EVENT

At the time of this event, Unit 2 was operating in Mode 1 (power operation) at 53 percent of rated thermal power. Other than that described herein, there was no inoperable equipment that contributed to the occurrence of this event.

#### C. DESCRIPTION OF EVENT

On October 14, 1996, power ascension was in progress following a refueling outage. Control room operators were manually exercising steam generator (SG) #3 main feedwater regulating valve (MFRV) 2FV-0530. The valve had been exhibiting sluggish and erratic responses to demands for both opening and closing. At 1920 EDT, the MFRV went from being mostly open to the 25 percent open position and SG #3 water level began to decrease when the valve would not open further. Although the bypass feedwater regulating valve (BFRV) was opened to supply SG #3, its flow capacity is unable to maintain water level in the SG at this power level. Operators were unsuccessful in attempts to re-open the MFRV and SG #3 water level continued dropping. By 1925 EDT, SG #3 level had dropped to

43 percent (narrow range) and a manual reactor trip was initiated.

Isolation of the main feedwater system and actuation of the auxiliary feedwater (AFW) system occurred as designed. Operators stabilized SG water levels and transitioned the unit to normal operation in Mode 3 (hot standby).

#### D. CAUSE OF EVENT

The cause of this event was a failure of the MFRV positioner mechanism. Personnel found that a valve stem inside the positioner's pilot valve was slightly worn and misshapen. The investigation is continuing to determine if any other anomalies contributed to the erratic valve responses.

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#### E. ANALYSIS OF EVENT

Control room operators properly responded to the decline in SG water levels by manually tripping the reactor. AFW actuated to supply water to the steam generators and operators again responded properly to control AFW flow and regain normal SG water levels. No problems arose following the trip that prevented operators from transitioning the plant to stable operation in Mode 3. Based on these considerations, there was no adverse effect on plant safety or on the health and safety of the public as a result of this event.

#### F. CORRECTIVE ACTIONS

- 1) The MFRV positioner was replaced and the valve was returned to

service.

2) The original positioner is being sent to the vendor for further investigation into the cause of the valve stem anomalies and their potential for causing the valve's erratic responses. The plan to implement the corrective actions that result from this investigation will be completed by March 17, 1997.

#### G. ADDITIONAL INFORMATION

1) Failed Components:

Valve positioner manufactured by Bailey Controls.

Model #AV1-12000

2) Previous Similar Events:

None

3) Energy Industry Identification System Code:

Main Feedwater System - SJ

Auxiliary Feedwater System - BA

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Georgia Power

C. K. McCoy the southern electric system

Vice President, Nuclear

Vogtle Project

November 8, 1996

LCV-0905

Docket No. 50-425

U. S. Nuclear Regulatory Commission

ATTN: Document Control Desk

Washington, D. C. 20555

Ladies and Gentlemen:

VOGTLE ELECTRIC GENERATING PLANT

LICENSEE EVENT REPORT 2-96-6

REACTOR TRIP DUE TO

MAIN FEEDWATER REGULATING VALVE CLOSURE

In accordance with the requirements of 10 CFR 50.73, Georgia Power

Company (GPC) hereby submits the enclosed report associated with an event  
that occurred on October 14, 1996.

Sincerely,

C. K. McCoy

CKM/TEW/AFS

Enclosure: LER 2-96-6

cc: Georgia Power Company

Mr. J. B. Beasley, Jr.

Mr. M. Sheibani

NORMS

U. S. Nuclear Regulatory Commission

Mr. S. D. Ebnetter, Regional Administrator

Mr. L. L. Wheeler, Licensing Project Manager, NRR

Mr. C. R. Ogle, Senior Resident Inspector, Vogtle

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